



Nuclear Process & Radio-Analytical Chemistry

Class Notes – Section 3.2

PUREX Process

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Major Plant Operating Periods

Fuel Reprocessing

T-Plant	1944 - 1956
B-Plant	1945 - 1952
REDOX	1952 - 1967
PUREX	1956 - 1972 and 1983 - 1988

Nuclear Materials Processing

Fuel Fabrication	1944 - 1987
Plutonium Finishing Plant	1949 -
U-Plant Uranium Recovery	1952 - 1957
UO₃ Plant	1952 - 1972 and 1984 - 1993



Redox reagents for used in nuclear process industry

Strong oxidizing agents

($\text{Pu}^{+3}/\text{Pu}^{+4} \rightarrow \text{Pu}^{+6}$):

- 1) $\text{K}_2\text{Cr}_2\text{O}_7$
- 2) KMnO_4
- 3) Ag^{+2}
- 4) NaBiO_3 (Sodium Bismuthate)
- 5) Ce^{+4}



Redox reagents for used in nuclear process industry

Strong reducing agents ($\text{Pu}^{+4} \rightarrow \text{Pu}^{+3}$):

- 1) Zn metal
- 2) Fe^{+2}
- 3) NaI (I^{-1})
- 4) $\text{NH}_2\text{OH}.\text{HNO}_3$ (hydroxylamine nitrate)
- 5) Ti^{+3} & U^{+4}



Common Oxidation States

In $\text{HNO}_3/\text{HNO}_2$ solution, majority of each actinide will be shown the following oxidation states:

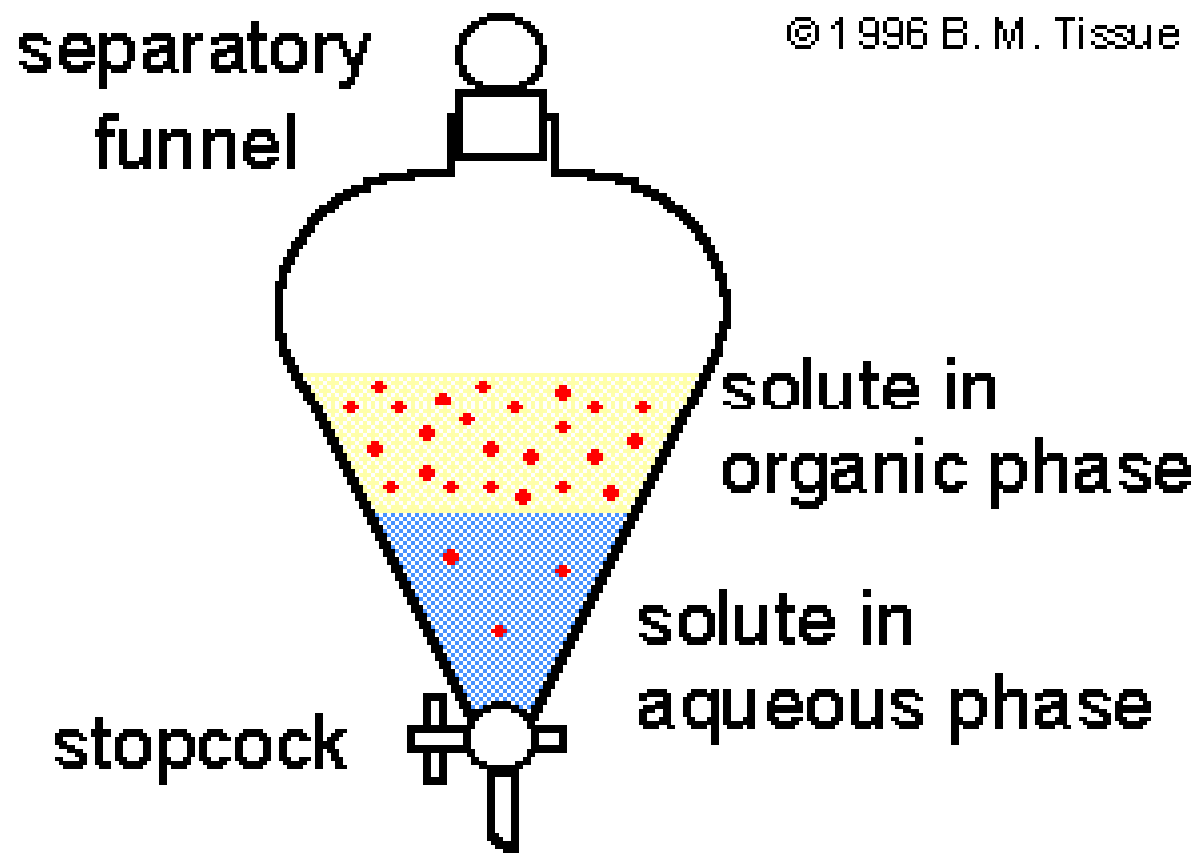
Actinide	Oxidation #
Am	+3
Pu	+4
Np	+5
U	+6



Solvent Extraction Behavior for Oxidation # of actinide

Oxidation #	Hexone	Tri-butyl Phosphate	Aliphatic Amine
+3	Non-extract	Non-extract	Non-extract
+4	Non-extract	extract	extract
+6	extract	extract	extract

Solvent Extraction





PUREX Process

PUREX is a liquid-liquid solvent extraction process that:

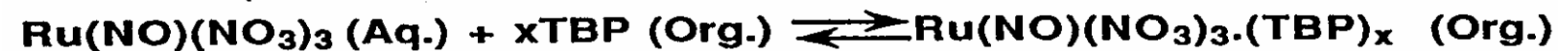
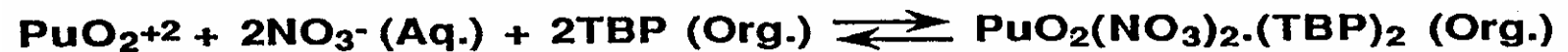
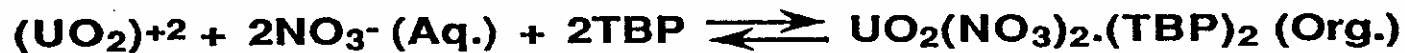
- 1) Separates Uranium and Plutonium from fission products in dissolved, irradiated reactor fuel; and then**
- 2) Separate Uranium from Plutonium.**

Solvent extraction is accomplished using tributyl phosphate (TBP) dissolved in a hydrocarbon diluent (NPH) at a concentration of 30 volume % TBP.



PUREX Process

PROCESS CHEMISTRY BASIS FOR THE PUREX SOLVENT EXTRACTION FLOWSHEET



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PUREX Process

EFFECT OF NITRIC ACID ON DISTRIBUTION COEFFICIENTS for URANIUM AND PLUTONIUM (in 20% TBP in hydrocarbon)

Metal or Radical	Distribution Coefficient	
	1 <u>M</u> HNO ₃	6 <u>M</u> HNO ₃
UO ₂ ++	5	30
PuO ₂ ++	0.7	3
Pu 4+	1.3	20
Pu 3+	0.015	0.01



PUREX Process

BEHAVIOR OF FISSION PRODUCTS IN EXTRACTION OF URANIUM FROM FISSION PRODUCTS [15 vol% TBP]

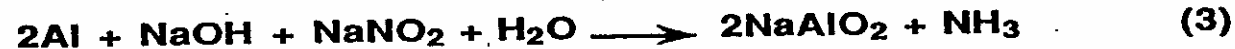
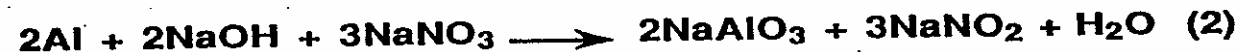
	DECONTAMINATION FACTOR		
	EXTRACTING SECTION	SCRUB SECTION	OVERALL
GROSS BETA	330	30	9.8 E3
RUTHENIUM	106	5	530
CERIUM	1.4 E3	380	5.3 E5
ZIRCONIUM	71	210	1.5 E4
NIوبيUM	1.5 E3	5.6	8.3 E3



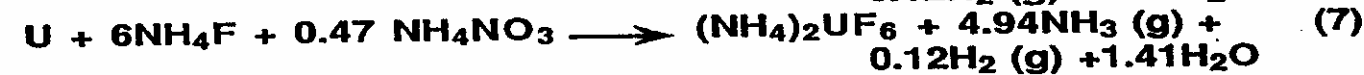
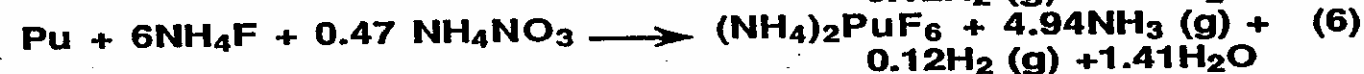
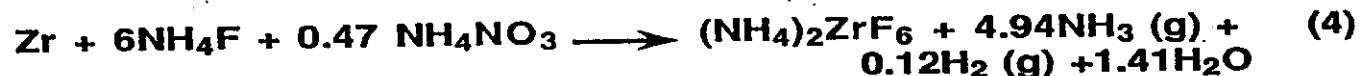
PUREX Process

IRRADIATED FUEL ELEMENT CHEMICAL DECLADDING

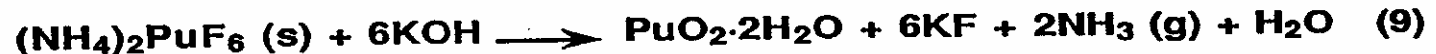
ALUMINUM JACKET REMOVAL



ZIRCALOY JACKET REMOVAL



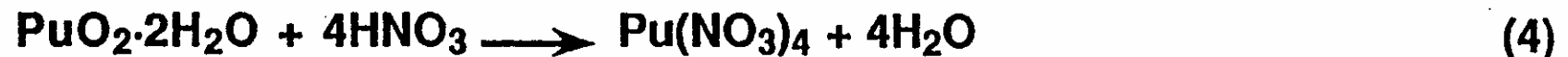
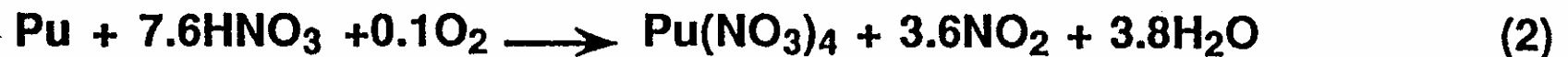
Metathesis



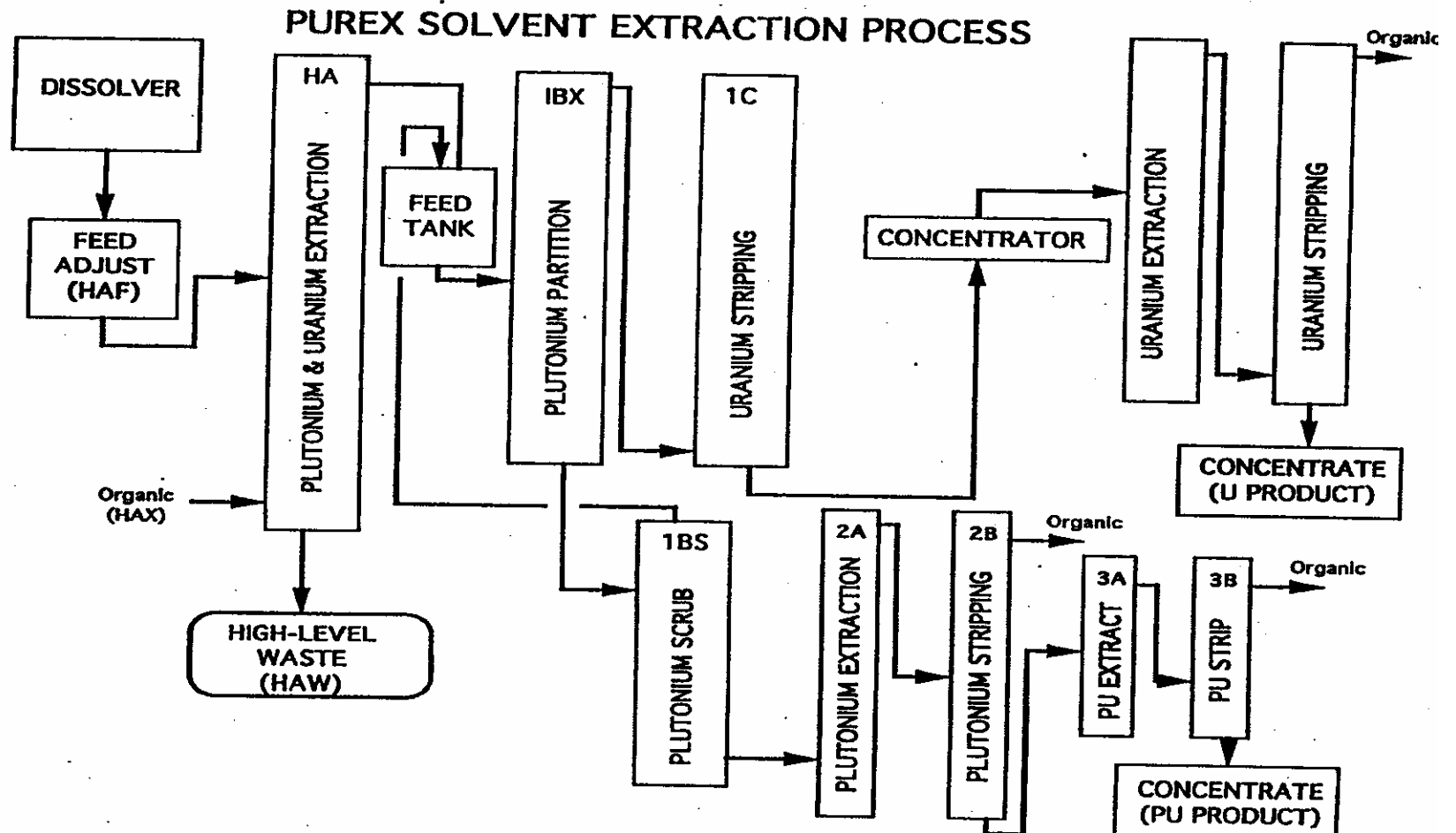


PUREX Process

FUEL DISSOLUTION



PUREX Process



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PUREX Process

